scientific reports



OPEN Thermodynamic model of the oxidation of Ln-doped UO₂

V. L. Vinograd[™], A. A. Bukaemskiy, G. Deissmann & G. Modolo

Ln-doped UO2 is often considered as a model system of spent nuclear fuel (SNF) helping to reveal effects of fission and activation products on its chemical stability. Comparing thermodynamics of UO2-UO3 and LnO1 5-UO2-UO3 systems provides a means to understand the phenomenon of an increased resistivity of Ln-doped UO₂ to oxidation in air relative to pure UO₂. Here a thermodynamic model is developed and is applied to investigate detailed phase changes occurring along the oxidation of Ln-doped fluorite to U_3O_8 . The study proposes that an enhanced resistivity to oxidation of Ln-doped UO₂ is likely caused by a thermodynamically driven partitioning of Ln between a fluorite-type phase and a U₃O₈ polymorph, which at ambient temperatures becomes hindered by slow diffusion.

Rare earth elements (REE) such as Nd, Ce, La and Y, occur as high yield fission products in spent nuclear fuel (SNF)^{1,2}, while Gd is often added to the fuel as a burnable poison³. The occurrence of these elements, colloquially referred here to as "lanthanides" (Ln), is very important for understanding thermodynamic differences between SNF and pure UO₂. These differences may have a profound influence on the chemical stability of SNF in diverse scenarios relevant to its final disposal in a deep geological repository. Although any repository is designed to prevent a contact of spent fuel with ground waters⁴, a possibility of such an interaction is expected after a long time span of ~ 10,000 years. This consideration motivates research on SNF dissolution. Under reducing conditions, pure UO₂ fluorite has a very low solubility in aqueous solutions⁵. The solubility of UO₂ is enhanced, however, in the presence of radiolytic oxidants, such as H₂O₂, which could be generated at the solid/water interface by alpharadiation. The high chemical reactivity of H₂O₂ then creates locally oxidizing conditions, at which much more soluble oxidized forms of UO₂ could be formed⁶. Many recent studies of oxidative dissolution of pure UO₂ and doped UO₂ solid solutions, including simulated fuels⁷⁻⁹, were concerned with the question of whether the fission products and transuranium elements that are contained in SNF enhance or suppress its oxidative dissolution rate in aqueous media. These studies clearly showed that doped UO2 dissolves in aqueous solutions at a much lower rate compared to pure UO2. Consistently, experiments on electrochemical dissolution of Ln-doped UO2 and of simulated fuels¹⁰⁻¹³ showed that impurities make UO₂ significantly less susceptible to oxidation. Also, considerable experimental evidence was accumulated showing that Ln-doping and burn-up significantly hinders the kinetics of oxidation of UO₂ and of UO₂-based SNF in air ¹⁴⁻¹⁸. Detailed analysis of this data ¹⁹ suggested that air-oxidation and oxidative dissolution studies performed with pure UO2 would provide conservative kinetic constraints for the reactivity of a spent UO₂-based nuclear fuel, while fuel dissolution rates determined with average burnup fuel would be also valid for high burnup fuel. Clearly, these important conclusions would have an even higher value for fuel performance assessments if the mechanism of the enhanced resistivity of doped UO2 was fully understood.

However, this mechanism is still discussed controversially. Razdan & Shoesmith¹¹ linked the stabilizing effect of Ln-doping to the formation of Ln-V clusters (V denotes oxygen vacancies), which were thought to reduce the number of vacant sites that could host oxygen anions. The formation of Ln-V clusters in reduced Ln-doped solid solutions (Ln = Y, Dy, Gd, Eu, Sm) was confirmed in a computational study²⁰. However, the active role of vacancies in the retardation effect is doubtful as the vacancies must be filled in before a sample could be oxidized to hyper-stoichiometry. Casella et al. 21 suggested that Ln^{3+} dopants due to their effectively negative charge (relative to U⁺⁴) repel oxygen interstitials thus limiting the fraction of interstitial sites available for accommodating an excess of O-2. The electrostatic origin of the retardation is doubtful, however, as the effect is also observed for Th⁺⁴ ²². Kim et al. ¹⁶ proposed that the decreased rates of oxidation in air measured on Gd-doped samples were due to a decreased fraction of U^{+4} caused by the reaction $2U^{+4} = Gd^{+3} + U^{+5}$. The decrease in the fraction of oxidizable U⁺⁴ was thought to hinder the incorporation of interstitials. The common difficulty of the discussed hypotheses^{11,16,21} is that the dopants are assumed to alter the lattice of fluorite locally hindering its ability to incorporate oxygen. This assumption implies the oxidative resistivity to be about linearly proportional to the

Institute of Energy and Climate Research—Nuclear Waste Management (IEK-6), Forschungszentrum Jülich GmbH, Jülich, Germany. [™]email: v.vinograd@fz-juelich.de

dopant concentration. On the contrary, oxidative dissolution yields drop by orders of a magnitude upon an addition of a few mole percent of a dopant 8,9.

Solution calorimetry studies^{23,24} provided evidence for a remarkable thermodynamic stability of Ln-doped UO₂ solid solutions with an oxygen to metal (O/M) ratio of ~ 2. The formation enthalpy of $U_{1-z}Ln_2O_2$ compounds (Ln = La, Nd, Y) becomes strongly more negative with the Ln fraction, z, for 0 < z < 0.5. The stability of these compounds correlates with an independent observation that Ln-doped solid solutions of intermediate compositions (0.5 < z < 0.67) synthesized in strongly oxidizing conditions at temperatures of about 1273 K after achieving the stoichiometric relationship of O/M ~ 2 do not oxidize further to hyper-stoichiometry²⁵⁻³⁰. Based on the measured endothermic energies Mazeina et al.²³ suggested that the activity of the UO₂ endmember is greatly reduced in doped fluorite solid solutions making them less prone to oxidation. However, this activity estimation assumed the applicability of a regular mixing model to UO₂- $LnO_{1.5}$ solid solutions. This assumption needs further testing.

A further understanding of the resistivity mechanism could possibly be found within a model that considers not only the fluorite phase, but also other phases that form because of its oxidation. Kinetic factors associated with $UO_2 \rightarrow U_4O_9$ and $U_4O_9 \rightarrow U_3O_8$ transformations could be then considered. Particularly, a coupling between thermodynamic and kinetic factors can be investigated. Therefore, here we compare the thermodynamics of oxidation in pure UO_2 and in Ln-doped systems, analyse changes in phase relations induced by the doping and discuss kinetic factors that would likely be enhanced in the doped case. The model is then applied to air oxidation experiments. Oxidative dissolution experiments are also discussed; however, no attempt is made to simulate processes in the aqueous phase.

Consequently, the aim here is to model phase changes occurring in UO_2 - UO_3 and $LnO_{1.5}$ - UO_2 - UO_3 systems along with changes in the temperature, T, the partial pressure of oxygen, PO_2 , and the mole fraction, z, of $LnO_{1.5}$. The assessment of $17 LnO_{1.5}$ - UO_2 - UO_3 systems (Ln = La - Lu, Y, Sc) is a too ambitious task. Conversely, considering only a single system, e.g., Ln = La, is also not a good option because the thermodynamic data would not be sufficient for developing a comprehensive model. Rather, we aim at a generalized model for a typical "Ln" that provides a reasonably good description of phase equilibria in many $LnO_{1.5}$ - UO_2 - UO_3 systems, being possibly deficient in some detail, in specific cases. Thus, we feed the model with data on different Ln-systems, assuming that differences in thermodynamic properties of lanthanides that are relevant to oxidation reactions can be ignored at a first glance. The emphasis is put on accurately reproducing the effect of Ln-doping on the equilibrium oxygen partial pressure by adjusting relative Gibbs free energies of relevant endmembers of solid solution phases. The gas phase is not simulated directly; the dependence on the oxygen partial pressure is formally included into the Gibbs free energies of solids. This simple approach is thought sufficient for the present task of evaluating the general effect of a trivalent dopant. The model derivation is mostly built on data on $Ln = \{La, Nd, Gd\}$. The data on heavier Ln are relatively scarce. We also exclude systems with Eu, Ce and Pr since these elements can occur in different oxidation states, making a simple generalized approach not feasible.

The study includes phases with $MO_{2+\delta}$, M_4O_9 , and M_3O_8 composition. The rhombohedral phases, M_7O_{12} , M_8O_{15} and M_8O_{16} , are omitted. Only the $Fm\overline{3}m$ $MO_{2+\delta}$ phase, colloquially referred here to as "fluorite", is treated as a non-stoichiometric compound. Because the model performance is illustrated here mostly with data on La and Nd, only the hexagonal (A) polymorph of M_2O_3 is considered. The tetragonal U_3O_7 phase is also omitted. Tetragonal phases with O/M ratios of ~ 2.3 and ~ 2.33 appear in oxidation experiments with pure UO_2 at temperatures below ~ 823 K 31 . In doped systems the tetragonal phases are observed rarely. However, a cubic γ - M_4O_9 phase with an O/M ratio of ~ 2.4 appears within the same stability range 14 . The present study does not make an attempt to exactly reproduce these complex phase relations. Instead, the observed tendency for the formation of fluorite-like phases with O/M ~ 2.33 is emulated here with the aid of the general fluorite model. The current fluorite phase model builds upon the previous model developed in 32 . The non-stoichiometry interval for $MO_{2+\delta}$ is extended to $-1/2 < \delta < 1/3$. As in 32 , an effort is made not only to describe the Gibbs free energy of the fluorite phase as a function of T, P_{O_2} , and z, but also to evaluate its lattice parameter as a function of the same variables. This is achieved by coupling the thermodynamic model with an ion-packing model. A detailed formulation is given in the "Methods" section. The results are presented below as a series of phase diagrams.

Results

Non-stoichiometry in the fluorite solid solution

The fluorite phase, $U_{1-z}Ln_zO_{2+\delta}$ is modelled here as a mixture of the endmembers UO_2 , $LnO_{1.5}$, $UO_{2.5}$, $U_{1/2}Ln_{1/2}O_{2}$, and $U_{1/3}Ln_{2/3}O_{2}$ (see "Methods"). The model appears to be sufficiently robust to describe the dependence of the non-stoichiometry parameter, δ , on T, z, and P_O . Figure 1 shows the model fit to the data for the pure UO₂-UO₃ system. Figure 2 shows the fit to the data for the GdO_{1.5}-UO₂-UO₃ system, where the composition, z, is varied within the range of 0 < z < 0.7. The fit to ΔG_{O_2} vs. δ , where $\Delta G_{O_2} = RT \ln(P_{O_2}/P^0)$; $P^0 = 101325$ Pa, is improved significantly relative to 32 , particularly, at z > 0.5. This is the result of extending the interval of δ to $(-1/2 < \delta < 1/3)$ and of including the new endmember $U_{1/3}Ln_{2/3}O_2$. The model parameters are given in Table 1. Importantly, some of the fitted parameters appear to be close to the values estimated from the data on UO2, LnO1.5, y-UO3 and U3O8 with the additivity rule (see Table S1 in Supplementary materials). This is observed, for example, in the case of $UO_{2,5}$, but not in the cases of $U_{1/2}Ln_{1/2}O_2$ and $U_{1/3}Ln_{2/3}O_2$. The fitted standard free energies of the latter endmembers (-76.0 and -72.0 kJ/mol, respectively) appear to be significantly lower than the corresponding estimates based on additivity. This implies that these endmembers are significantly stabilized due to certain interactions occurring between $UO_{2.5}$ and $LnO_{1.5}$ and between UO_3 and $LnO_{1.5}$. A good fit was obtained by defining Margules parameters only for UO2-UO2.5 and UO2-GdO1.5 interactions. All other interactions were set athermal. The same model provided a reasonably good fit to $\Delta G_{\rm O}$, vs. δ data for systems with $NdO_{1.5}^{33}$ and $LaO_{1.5}^{34,35}$. This is illustrated in Figs. S1 and S2 (Supplementary materials). Thus, practically, the model is applicable to systems with $Ln = \{La, Nd, Gd\}$ with no modification.

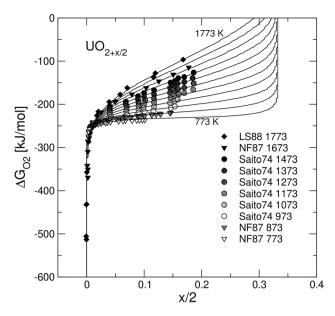


Figure 1. Model fit to the experimental data on $\Delta G_{\rm O_2} = RT \ln(P_{\rm O_2}/P^0)$ vs. $\delta = x/2$ for pure UO₂. The experimental data are from Lindemer & Sutton³⁶, Nakamura & Fujino³⁷ and Saito³⁸.

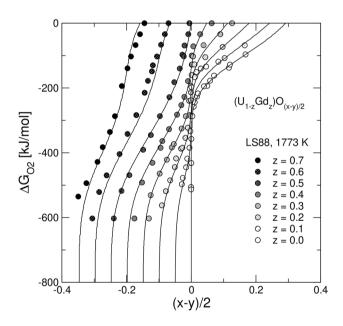


Figure 2. Model fit to the experimental data on $\Delta G_{\rm O_2} = RT \ln(P_{\rm O_2}/P^0)$ vs. $\delta = (x-y)/2$ for the system ${\rm GdO_{1.5}\text{-}UO_2\text{-}UO_3}$. The experimental data are from Lindemer & Sutton³⁶. Note, that δ can be written as $\delta = (x-y)/2$, as it is a function of the mole fractions, x and y, of the UO_{2.5} and $LnO_{1.5}$ endmembers. δ can be also evaluated as $\delta = O/M - 2$. A negative/positive deviation from $\delta = 0$ implies the presence of either oxygen vacancies or oxygen interstitials. Note also, that δ characterizes non-stoichiometry only of a monophase fluorite, while O/M - 2 is also applicable to a poly-phase system. $\Delta G_{\rm O_2}$ is a convenient function to visualize effects of the temperature and/or the partial pressure of O_2 on the chemical potential of O_2 , while the dimensionless quantity $\log(P_{\rm O_2}/P^0)$ is more convenient when the temperature and the pressure effects need to be distinguished. Both quantities are used throughout the text.

Oxidation of pure UO2

The equilibrium oxidation of pure UO_2 at 973–1273 $K^{38,39}$ occurs through the following sequence of transformations. First, the O/M ratio increases within the mono-phase fluorite, F, reaching a certain limiting value, then, a two-phase $F + U_4O_9$ mixture develops. After the fluorite phase in the mixture becomes extinct, a mono-phase U_4O_9 remains stable within a certain interval of $log(P_{O_2}/P^0)$, and, finally, a two-phase assemblage with varying

Fluorite solid solution	(cubic)		
Endmember	i	$\Delta G_i^0(kJ/mol)$	$\Delta S_i^0(J/K/mol)$	$\Delta Cp_i^0(J/K/mol)$
UO ₂	1	0.0	0.0	0.0
LnO _{1.5}	2	0.0	0.0	0.0
UO _{2.5}	3	-81.025	7.0	8.5
$Ln_{1/2}^3{\rm U}_{1/2}^5{\rm O}_2$	4	-76.0	0.0	4.25
$Ln_{2/3}^3 U_{1/3}^6 O_2$	5	-72.0	3.5	6.0
Margules parameter	i,j	$W_{ij}^h(kJ/mol)$	W ^s _{ij} (J/K/mol)	
	1,2	28.0	0.0	
	1,3	28.3	27.75	
U ₄ O ₉ solid solutions (c	ubic, β	or γ)		
Endmember	i	$\Delta G_i^0(kJ/mol)$	$\Delta S_i^0(J/K/mol)$	$\Delta Cp_i^0(J/K/mol)$
$U_{1/2}^4 U_{1/2}^5 O_{9/4}$	1	-37.75	8.7	6.0
$\{Ln_{1/4}^3 U_{1/4}^5\} U_{1/2}^5 O_{9/4}$	2	-80.3	6.37	3.0
Margules parameter	i,j	$W_{ij}^h(kJ/mol)$	$W_{ij}^{s}(J/K/mol)$	
	1,2	0.0	0.0	
$\alpha\text{-}U_3O_8$ solid solution (ortho	hombic)		
Endmember	i	$\Delta G_i^0(kJ/mol)$	$\Delta S_i^0(J/K/mol)$	$\Delta Cp_i^0(J/K/mol)$
$U_{2/3}^5 U_{1/3}^6 O_{8/3}$	1	-95.3	11.21	15.7
$\{Ln_{2/9}^3 U_{4/9}^6\} U_{1/3}^6 O_{8/3}$	2	-97.7	14.84	14.05
Margules parameter	i,j	$W_{ij}^h(kJ/mol)$	$W_{ij}^s(J/K/mol)$	
	1,2	5.0	0.0	
α '- U_3O_8 solid solution	(hexag	onal)		
Endmember	i	$\Delta G_i^0(kJ/mol)$	$\Delta S_i^0(J/K/mol)$	$\Delta Cp_i^0(J/K/mol)$
$\{U_{2/3}^5U_{1/3}^6\}O_{8/3}$	1	-92.0	11.21	15.7
$\{Ln_{2/9}^3 \mathbf{U}_{7/9}^6\} \mathbf{O}_{8/3}$	2	-97.7	14.84	14.05
Margules parameter	i,j	W ^h _{ij} (kJ/mol)	W ^s _{ij} (J/K/mol)	
	1,2	5.0	0.0	
A-Ln ₂ O ₃ (hexagonal)				
LnO _{1.5}	1	3.3	7.5	0.0

Table 1. Adopted thermodynamic parameters for the solid solution phases. Cations taken in curly brackets contribute to the configurational entropy of an endmember. This contribution is not included in the standard entropy values of the endmembers given in the Table but is counted within the general equation describing the entropy of mixing of a relevant phase. Upper symbols in structural formulas denote charges of cations.

fractions of U_4O_9 and U_3O_8 evolves, reaching an O/M ratio of ~ 2.6. The sequence $F \rightarrow F + U_4O_9 \rightarrow U_4O_9 \rightarrow U_4$ $O_9 + U_3 O_8 \rightarrow U_3 O_8$ is successfully reproduced in Fig. 3. The construction of the diagram required defining the standard Gibbs free energies of the endmembers. $U_{1/2}^4U_{1/2}^5O_{9/4}$ and $\{U_{2/3}^5U_{1/3}^6\}O_{8/3}$ that correspond to γ -U₄O₉ and to the hexagonal α'-U₃O₈ polymorphs, respectively. At temperatures below 873 K the model predicts a reappearance of the fluorite phase, F2, with $\delta \sim 0.33$, which at O/M > 2.33 co-exists with U_3O_8 . The sequence of transformations below 873 K is thus F1 \rightarrow F1 + U₄O₉ \rightarrow U₄O₉ \rightarrow U₄O₉ + F2 \rightarrow F2 \rightarrow F2 + U₃O₈ \rightarrow U₃O₈. The symbols F1 and F2 distinguish different states within the same fluorite phase. The transformation $U_4O_9 \rightarrow F2$ is driven by an increase in the chemical potential of oxygen caused by the decrease in temperature. The stabilization of F2 fluorite with $\delta \sim 0.33$ and the disappearance of U_4O_9 ($\delta = 0.25$) is the consequence of a larger δ value achievable in fluorite. The model allows to qualitatively explain the experimentally observed appearance of fluorite-like tetragonal phases with $\delta \sim 0.33$ at $T < 823^{31}$. Although the tetragonal β -U₃O₇ phase differs structurally from fluorite⁴⁰, its thermodynamic properties are likely similar to F2 fluorite with $\delta \sim 0.33$. Indeed, the data of Grenthe et al. 41 indicate that U₃O₇ is stable just by 0.7 ± 2.0 kJ/mol relative to the mixture of UO_{2.25} and UO_{2.67}. Similarly, in our simulations, F2 fluorite ($\delta \sim 0.33$) is marginally stable relative to the same phases at temperatures below 873 K. At ~ 1373 K U_4O_9 becomes unstable relative to the F1 phase (with δ ~ 0.25). Thus, above 1373 K the phase sequence simplifies to $F1 \rightarrow F1 + U_3O_8$.

Oxidation of Ln-doped fluorite

Figure 4 shows the fit to the data of Stadlbauer et al.²⁹ for UO₂ doped with 4.8 mol % of LaO_{1.5}. The fit required defining the standard Gibbs free energies of the endmembers $\{Ln_{1/4}^3U_{1/4}^5\}U_{1/2}^5O_{9/4}$ and $\{Ln_{2/9}^3U_{7/9}^6\}O_{8/3}$ for the γ - and α -phases, respectively. Stadlbauer et al.²⁹ concluded that the phase relations in the doped system remain essentially the same as in the pure system. Thus, one would expect to see the phase sequence F1 \rightarrow F1 + M₄O₉ \rightarrow M₄O₉ \rightarrow M₄O₉ \rightarrow M₄O₉ \rightarrow M₃O₈. However, the predicted phase relations (Fig. 4) are more complicated at high O/M ratios. At O/M \sim 2.60 fluorite reappears, and the sequence is modified as: F1 \rightarrow F1 + M₄O₉ \rightarrow M₄O₉ \rightarrow

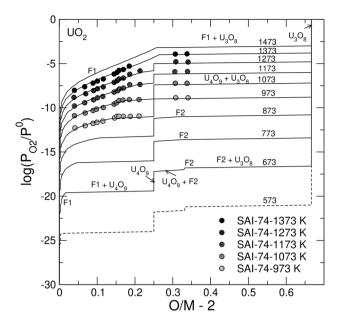


Figure 3. Model fit to the experimental data on $log(P_{O_2}/P^0)$ vs. O/M-2 for pure UO₂. The experimental data are from Saito ³⁸. Solid lines correspond to the equilibrium with the hexagonal polymorph of U₃O₈. The dashed line corresponds to the orthorhombic α-U₃O₈.

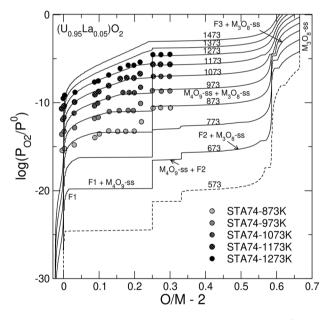


Figure 4. Model fit to the experimental data on $\log(P_{\rm O_2}/P^0)$ vs. O/M-2 for UO₂ doped with 4.8 mol % of LaO_{1.5}. The experimental data are from Stadlbauer et al. ²⁹. Solid lines correspond to the equilibrium with the hexagonal polymorph of M₃O₈. The dashed line corresponds to the orthorhombic α-M₃O₈.

 $O_9 + M_3O_8 \rightarrow F3 + M_3O_8 \rightarrow M_3O_8$. Importantly, the F3 phase appearing at O/M = 2.60 differs significantly from the F1 phase occurring within the range of 0 < O/M < 2.25. This F3 phase is very Ln-rich, while its O/M ratio is close to 2.0. Notably, the F2 phase with $\delta \sim 0.33$ is also stable in the doped system at T < 873. The most significant difference relatively to the pure system is the rise of the isotherms within the biphasic $M_4O_9 + M_3O_8$, $F2 + M_3O_8$ and $F3 + M_3O_8$ regions to higher oxygen partial pressures. Figures 5 and 6 investigate the corresponding changes in detail. The positive slope of the isotherms in Fig. 4 at O/M > 2.25 correlates with the increase in the Ln-content in the M_4O_9 phase (Fig. 5). As the O/M ratio increases, Ln accumulates in M_4O_9 , while M_3O_8 takes a negligeable part of the total LnO_2 . The strong partitioning of Ln into M_4O_9 can be attributed to the stability of the $\{Ln_{1/4}^3 U_{1/4}^4\} U_{1/2}^5 O_{9/4}$ endmember. The assessed standard Gibbs free energy of the $\{Ln_{1/4}^3 U_{1/4}^4\} U_{1/2}^5 O_{9/4}$ end-

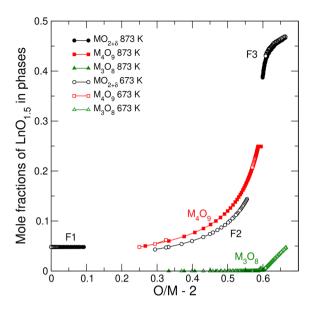


Figure 5. Predicted evolution of composition of phases in a sample containing 4.8 mol % of LaO_{1.5} in the process of equilibrium oxidation at 673 and 873 K. Filled and empty symbols correspond to 873 K and 673 K isotherms, respectively.

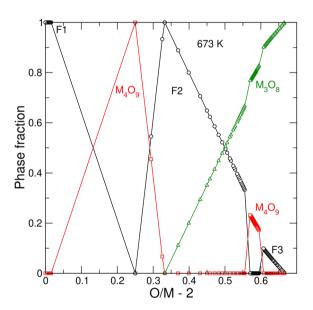


Figure 6. Fractions of phases in a sample containing 4.8 mol % of LaO $_{1.5}$ in the process of equilibrium oxidation at 673 K. Symbols are the same as in Fig. 5.

member (Table 1), is approximately equal to the half sum of the free energies of UO_{2.5} and U_{1/2} $Ln_{1/2}$ O₂. Thus, the high stability of U_{1/2} $Ln_{1/2}$ O₂ component contributes to the stability of { $Ln_{1/4}^3U_{1/4}^5$ }U_{1/2}⁵O_{9/4}. As log($P_{\rm O_2}/P^0$) increases, the fraction of M₃O₈ grows up while the fractions of M₄O₉ and/or F2 phases decrease (Fig. 6). This change occurs because a high total O/M ratio (e.g., $O/M \sim 2.6$) could only be achieved when the fraction of M₃O₈ is large. Because M₃O₈ is nearly free of Ln in the association with M₄O₉, the concentration of LnO_{1.5} within the minor M₄O₉ phase increases with O/M and quite rapidly reaches the theoretical limit of z = 0.25. After the M₄O₉ and F2 phases vanish, fluorite remains as the F3 phase. The content of Ln in the F3 phase rapidly increases to ~45 mol %. This implies that the equilibrium oxidation of the doped sample containing 4.8 mol % of LaO_{1.5} requires a nine-fold enrichment of the fluorite phase in LnO_{1.5}.

The ratio of $O/M \sim 2.0$ within the Ln-rich $\bar{\rm F3}$ phase – the consequence of an exceptionally strong stability of the ${\rm U_{1/2}}Ln_{1/2}{\rm O_2}$ endmember – has an important impact on phase relations at high $P_{\rm O_2}$. When $\log(P_{\rm O_2}/P^0)$ approaches zero, the partitioning of Ln into the ${\rm M_3O_8}$ phase becomes favourable (Fig. 5). This partitioning is driven by the tendency of the system to achieve the largest total O/M ratio of ~ 2.67 , which would be impossible

to attain if the stoichiometric F3 fluorite ($\delta \sim 0.0$) remained as a significant fraction in the system. The increase in the total O/M at O/M > 2.6 thus causes a decrease in the fraction of F3 fluorite in the mixture and an increase in the concentration of Ln in M_3O_8 (Figs. 5 and 6). The final state of a fully oxidized Ln-doped UO_2 is a monophase Ln-doped M_3O_8 . This prediction remains valid only at temperatures below ~ 973 K. As the temperature increases, the chemical potential of oxygen decreases, and the final assemblage is biphasic, consisting of an Ln-poor M_3O_8 and a very Ln-rich nearly stoichiometric fluorite. Figures S3 and S4 (Supplementary materials) illustrate the equilibrium oxidation at 10 mol % of LnO_{15} .

Lattice parameter variation as a function of oxygen partial pressure, temperature, and Ln-concentration

The lattice parameter, a, of fluorite depends on z and on δ . This dependence is modelled here via an ion-packing model which is linked to the thermodynamic model. Figure 7 plots the equilibrium values of a in the system of NdO_{1.5}-UO₂-UO₃ computed at 1123 K and at $\log(P_{\rm O_2}/P^0)$ varying in the range [-30, -2]. Additionally, the isotherms of 1373 K and 1673 K are plotted at $P_{\rm O_2}=P^0$. Hyper-, hypo- and strictly stoichiometric states can be distinguished. Flat regions in the predicted variation of a vs. z correspond to two-phase assemblages, where fluorite co-exists with M_3O_8 or $A-Nd_2O_3$ phases. The two-phase co-existence of fluorite and M_4O_9 is represented as a rectangle. The diagram predicts that M_4O_9 phase is stable at 1123 K within the interval of $-8 < \log(P_{\rm O}/P^0) < -4$. Thus, within this interval the phase sequence with the increase in z is as follows: $M_3O_8 + M_4O_9 \rightarrow M_4O_9 + F \rightarrow F \rightarrow F + A-Nd_2O_3$. At a higher P_{O_3} , the sequence is $M_3O_8 + F \rightarrow F \rightarrow F + A-Nd_2O_3$. An interesting feature is the change in the slope in the a vs. z relationship occurring in oxidized samples at $z \sim 0.67$. In strongly oxidized samples (i.e., at $\log(P_{\rm O_2}/P^0) \sim 0$, T = 1123 K), the change of slope correlates with a rapid ingrowth in the fraction of the $LnO_{1.5}$ component, i.e., with the fraction of vacancies. Thus, below and above the composition of z = 0.67 the fluorite phase is markedly different; within the interval of 0.5 < z < 0.67 fluorite closely maintains the stoichiometric relationship (O/M=2) – a feature, which reflects the stability of the endmembers $U_{1/2}Ln_{1/2}O_2$ and $U_{1/3}Ln_{2/3}O_2$, while at z > 0.67 it is hypo-stoichiometric. A similar change in the slope of the a vs. z relationship is also seen in the systems with La, Gd, Eu, and $Y^{25,27,42,43}$. The dependence of the lattice parameter in the system of LaO_{1.5}-UO₂-UO₃ is shown in Figure S5. Indeed, graphs analogous to Fig. 7 or Figure S5 can be easily constructed with the presently developed model for any $LnO_{1.5}$ - UO_2 - UO_3 system for which the ionic radii of Ln in 6-, 7-, and 8-fold coordination are known. Table S2 lists the relevant radii for systems with La, Nd, Gd and Y. The transformation at $z \sim 0.67$ suggests that in any oxidized $LnO_{1.5}$ - UO_2 - UO_3 system, where Ln is in +3 state, the solid composition at $z \sim 0.67$ is given by the formula $U_{1/3}^{6.8}Ln_{2/3}^{3.8}O_2$. This observation allows determining the ionic radius of an eightfold coordinated U^{+6} from $a = \frac{4}{\sqrt{3}} \left((2/3)R_{Ln}^{3.8} + (1/3)R_{U}^{6.8} + R_{O} \right)$ (see "Methods"). The value of $R_{U}^{6.8} = 0.0518^{-3}$ for the characteristic lates $R_{U}^{6.8} = 0.0518^{-3}$ for the characteristic lates $R_{U}^{6.8} = 0.0518^{-3}$ for the characteristic $R_{U}^{6.8} = 0.0518^{-3}$ for the characteristic $R_{U}^{6.8} = 0.0518^{-3}$ for $R_{U}^{6.8} = 0.0518^{-3}$ $0.755\ \text{\AA}$ fitted to this relationship appears to be considerably smaller than the Shannon's value of 0.86^{44} . This discrepancy could possibly mean that the Shannon's value corresponds to an uranyl-like environment, while the smaller value reflects the ionic radius of U⁺⁶ in the cubic coordination.

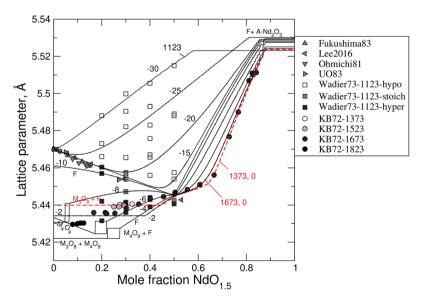


Figure 7. Variation of the lattice parameter in UO_2 -Nd $O_{1.5}$ solid solutions predicted from the thermodynamic model. Solid lines are the 1123 K isotherms computed at different values of $log(P_{O_2}/P^0)$ such that the pressure variation approximately covers the range of redox conditions in the data of Wadier ⁴⁵. Dashed lines (red online) are the 1373 and 1673 K isotherms computed at $log(P_{O_2}/P^0) = 0$. These isotherms correspond to the synthesis conditions in the study of Keller & Boroujerdi²⁶. The other experimental data are from Lee et al. ⁴⁶, Fukushima et al. ⁴⁷, Ohmichi et al. ⁴⁸ and Une & Oguma³³.

Discussion

The remarkable thermodynamic stability of compounds with the composition of $U_{0.5}Ln_{0.5}O_2$ has been previously reported based on solution calorimetry data^{23,24}. The formation energies of $U_{1-}Ln_zO_2$ (Ln = Y, Nd, La) relative to stable oxides (UO₂, UO₃ and $LnO_{1.5}$) fall onto a linear trend $\Delta H_{f,ox} = -(103.8 \pm 4.3)z$ kJ/mol²⁴. Using the values of $\Delta G_{\text{U}_{1/2}\text{Ln}_{1/2}\text{O}_2}^0 = -76.0 \text{ kJ/mol}, \ \Delta G_{\text{LnO}_{1.5}}^0 = 3.3 \text{ kJ/mol} \ (\text{Table 1}) \ \text{and} \ \Delta G_{\text{UO}_3}^0 = -113.9 \text{ kJ/mol} \ (\text{Table S1 in Supplementary materials}) we obtain for <math>Ln = \{\text{La, Nd}\}: \Delta G \sim \Delta H = (-76 - (0/4 - 113.9/4 + 3.3/2)) = -49.2 \text{ kJ/mol}, \text{ which } 1/2 \text{ kJ/mol} \ \text{which } 1/2 \text{ kJ/mol} \ \text{kJ/mol} \ \text{which } 1/2 \text{ kJ/mol} \ \text{kJ/mol} \$ agrees well with the solution calorimetry trend at z = 0.5, as well as with ab initio calculations⁴⁹. Considering the large endothermic formation energy of $U_{1/2}Ln_{1/2}O_2$ and $U_{1/3}Ln_{2/3}O_2$ endmembers, the tendency of samples with 0.5 < z < 0.67 to preserve stoichiometry in oxidizing conditions is well understood. The O/M = 2 appears to represent the maximum oxidation state of fluorite at z = 0.5. A further oxidation of the $U_{0.5}Ln_{0.5}O_2$ compound to hyper-stoichiometry would require the oxidation of U⁺⁵ to U⁺⁶, which would be associated with a significant lattice contraction. Such an oxidation/contraction is not observed experimentally, suggesting that U⁺⁶ might be too small to be easily accommodated into the fluorite lattice. On the other hand, when an excess of LnO_{15} over the $U_{0.5}Ln_{0.5}O_2$ composition is added to fluorite, the advantage of a further annihilation of vacancies makes the $U^{+5} \rightarrow U^{+6}$ exidation favourable; the system gains an equivalent amount of oxygen that is immediately consumed in the endothermic vacancy annihilation process. Consequently, the stoichiometric states extend to z = 0.67. This suggests that U^{+6} can be stable in fluorite only at an excess of $LnO_{1.5}$, i.e., when z > 0.5. A rapid increase in the lattice parameter at z > 0.67 can be attributed to the fact that at z > 0.67 oxygen vacancies cannot be avoided. The significantly larger effective size of the oxygen vacancy (see "Methods") relative to the ionic radius of oxygen explains the positive slope.

The tendency of Ln-doped UO_2 to keep $O/M \sim 2$ is also seen in Fig. 7 as the clustering in a vs. z data along the linear trend extending from stoichiometric UO_2 towards fully oxidized samples with $z \sim 0.5$. In Fig. 7 this stoichiometric trend is outlined as an attractor for 1123 K isochores computed at different partial pressures of oxygen. The convergence of isochores implies the stability of stoichiometric states within a wide interval of $\log(P_{O_2}/P^0)$. Figure S6 illustrates the evolution in the endmember fractions at 1273 K along the transition from reduced to oxidized samples. This evolution clearly shows the growing importance of the $U_{1/2}Ln_{1/2}O_2$ and $U_{1/3}Ln_{2/3}O_2$ components both with an increase in $\log(P_{O_2}/P^0)$ and with an increase in the mole fraction of $LnO_{1.5}$.

Consistently with the synthesis studies performed in oxygen or in air at $1173 < T < 1823 \text{ K}^{25,26}$, the present model predicts a co-existence of U_3O_8 with a fluorite phase, where the latter is significantly enriched in $LnO_{1.5}$ (Fig. 7). This fluorite phase is predicted to be closely stoichiometric. The M_4O_9 phase is stable in association with M_3O_8 only within a narrow interval of $\log\left(P_{O_2}/P^0\right)$. At highly oxidizing conditions M_4O_9 is destabilized relative to the fluorite (F3) phase. The reason for the destabilization is the partitioning of Ln into M_4O_9 coupled with the limited ability this phase to incorporate Ln^{+3} . Due to the same reason the M_4O_9 phase becomes unstable relative to fluorite even at a very small total $LnO_{1.5}$ fraction when the total O/M ratio is large (Figs. 5 and 6). Our study predicts, however, that at an excessively high chemical potential of O_2 (e.g., in air at temperatures below 973 K) the association of $F + M_3O_8$ becomes unstable relative to a mono-phase M_3O_8 system (Figs. 5 and 6). The reason for this instability is the tendency of the Ln-rich fluorite to be stoichiometric. Thus, to achieve a total $O/M \sim 2.67$, the system must get rid of fluorite completely. This prediction is consistent with the results of the recent study of Potts et al. 50 .

The comparison of Figs. 3 and 4 shows that in the doped case a higher oxygen pressure is required to oxidize the system to the same O/M ratio, i.e., the doped system is more stable against oxidation. However, within the range of 2.25 < O/M < 2.5, where the main retardation effect is thought to take place, the difference in $\log(P_{\rm O_2}/P^0)$ is less than one unit. Thus, it is unlikely that the oxidation resistance is a pure thermodynamic effect. Kinetic factors should be given a closer attention. The model predicts that at ~873 K in the doped system the equilibrium transformation goes along the sequence $F1 \rightarrow F1 + M_4O_9 \rightarrow M_4O_9 \rightarrow M_4O_9 + M_3O_8 \rightarrow F3 + M_3O_8 \rightarrow M_3O_8$, which includes several biphasic states. Figure 5 shows that these states involve partitioning of Ln between the phases, which is particularly strong in the cases of M₄O₉ + M₃O₈ and F3 + M₃O₈ co-existence. This partitioning implies that nucleation and growth of a nearly pure U_3O_8 from an Ln-bearing M_4O_9 requires redistributing $LnO_{1.5}$ back into the M_4O_9 phase. At T < 873 K the sequence of phases is modified to $F1 \rightarrow F1 + M_4O_9 \rightarrow M_4O_9 \rightarrow M_4O_9 + F$ $2 \rightarrow F2 \rightarrow F2 + M_3O_8 \rightarrow F3 + M_3O_8 \rightarrow M_3O_8$. The predicted stabilization of F2 in the doped system is important because this phase can possibly emulate the appearance of γ - M_4 O₉ described by Thomas et al. ¹⁴. It is conceivable that at a low temperature the redistribution of *Ln* between the phases would be controlled by the speed of solidstate Ln-diffusion. Likely, a crystallization of a small amount of U₃O₈ would be concurrent with the formation of a thin Ln-enriched layer in M_4O_9 or in γ - M_4O_9 , which would have protective properties; a continuing growth of U₃O₈ would require a further enrichment of this layer making it less compositionally suitable for re-crystallizing into a Ln-poor U₃O₈. Even at a low doping level such a mechanism would eventually cause the formation of a layer sufficiently enriched in Ln, from which the growth of Ln-poor U_3O_8 would be practically impossible. In this respect a pure system differs remarkably from any doped system. The oxidation of pure UO₂ requires oxygen diffusion only, which is known to be several orders of a magnitude faster than U self-diffusion in UO_2^{51} . On the other hand, a recent computational study⁵² showed that diffusion coefficients of U⁺⁴, La⁺³ and Y⁺³ differ only within an order of a magnitude, meaning that the *Ln* diffusion in UO₂ is many orders of magnitude slower than O-diffusion. This proposition could explain a surprisingly large effect of small loads of trivalent dopants on rates of UO₂ oxidation in air. The idea is that the resistance occurs not because the doping hinders the ability of fluorite and/or M₄O₉ to sorb oxygen, but because a dopant that forms a stable solid solution within fluorite or M₄O₉, cannot be easily transferred into a relatively *Ln*-poor more oxidised phase, i.e., U₃O₈, and must diffuse back into the parent cubic phase.

The proposed retardation mechanism is linked to the thermodynamic tendency of the cubic phases to be enriched in Ln relatively to orthorhombic or hexagonal U_3O_8 . As discussed above, at a very high oxygen potential

this partitioning scheme becomes less polarized, i.e., the concentration of Ln increases in the M_3O_8 phase, while the fraction of Ln-rich fluorite decreases in the system (Figs. 5 and 6). As oxidation experiments are typically performed at low temperatures in air where the Ln-rich M_3O_8 becomes stable, the proposed retardation mechanism involving Ln diffusion in M_4O_9 could be questioned. However, considering that the experiments are carried out with an excess of a preliminary reduced material, the effective partial pressure of oxygen at the reaction front should be significantly lower than the one in air. Likely, the oxygen pressure is buffered by the M_4O_9/M_3O_8 co-existence.

The retardation mechanism is discussed here mostly in the context of MO_2 oxidation in air. An adequate description of oxidative dissolution would require the modelling of an aqueous phase containing dissolved forms of U^{+6} . We could speculate, however, that an oxidation in an aqueous system would similarly include a dopant partitioning between an aqueous phase and an M_4O_9 -like phase. A recent oxidative dissolution study of an unirradiated homogeneous mixed oxide fuel (MOX) sample with the composition of $(Pu_{0.27}U_{0.73})O_2^{53}$ showed that the dissolution is incongruent, i.e., the ratio of the total dissolved Pu to the total dissolved U is about two orders of a magnitude lower than the Pu/U ratio in the solid, while the surface of the sample is enriched in Pu up to the composition of $(Pu_{0.39}U_{0.61})O_{2+\delta}$. The formation of a Th-enriched protective layer has been also inferred in an electrochemical study of oxidation in the $(U_{1-z}Th_z)O_2$ system⁵⁴. Thus, the dopant partitioning (and the diffusion of a dopant within an M_4O_9 -like phase), as a factor, cannot be excluded in dissolution experiments too. Clearly, a further understanding of the retardation effect would benefit from detailed spectroscopic and microanalytical studies of dopant partitioning in phases that crystallize at the oxidation front.

Methods

Thermodynamic approach

The modelling of LnO_{1.5}-UO₂-UO₃ systems is typically done with the CALPHAD methodology where solid solutions are described with the Compound Energy Formalism (CEF)⁵⁵⁻⁵⁷. In CEF endmembers are generated considering all possible occupations of available sublattices by admissible species⁵⁸. In the case of the fluorite model with four cations in the M sublattice and two anionic sublattices this procedure gives 16 endmembers, many of which are not of neutral charge. In CEF the Gibbs free energies of charged endmembers are carefully constrained such that in all equations they appear only in charge-neutral combinations⁵⁹. Thermodynamic functions of some individual endmembers may be chosen arbitrarily and their importance in determining real solid solution properties is difficult to visualise. The present model is more transparent. We employ only neutral endmembers and formulate site occupancies as linear functions of the endmember fractions. This means, for example, that the fractions of vacancies and O^{-2} interstitials in $MO_{2+\delta}$ fluorite are linked to the fractions of the neutral endmembers LnO_{1.5} and UO_{2.5}, respectively. We also allow for the presence of mixed endmembers, such as $Ln_{1/2}U_{1/2}O_2$ and $Ln_{2/3}U_{1/3}O_2$. Such endmembers cannot be easily included in CEF. Further, we introduce shortrange order (SRO) constraints; we reduce the randomness of cation and anion distribution within sublattices by limiting the space available for mixing of species to a subset of available sites. For example, when a vacancy is introduced into a fluorite MO2 solid solution, it is not allowed to substitute for any lattice oxygen. Rather, the oxygen-sublattice splits into two imaginary sublattices. One of them accommodates vacancies, while the other remains fully occupied by O⁻² anions. The fraction of sites over which the O/V mixing is allowed is considered as a model parameter. Decreasing this fraction emulates entropy reduction due to the vacancy/vacancy avoidance. Finally, we allow for a complete exclusion of certain species from mixing with other species. Particularly, in the fluorite model the fraction of U⁺⁵ cations that is needed to balance the fraction of O⁻² interstitials is excluded from mixing with other cations. Such an exclusion emulates local cation-anion association, causing an additional entropy reduction. The main advantage is that the number of parameters needed to be defined is decreased significantly compared to CEF.

These non-standard model features prevent us from using available Gibbs free energy minimization software. Thus, the calculations are performed with our own code. Several simplifications are adopted. The thermodynamic properties of all endmembers in all phases are defined relative to a mechanical mixture of a stoichiometric UO_2 fluorite and a hypothetical ordered $LnO_{1.5}$ with the pyrochlore, $Ln_2Ln_2O_6VV$, structure. The Gibbs free energies of these two endmembers are set to zero at all temperatures. The Gibbs free energy of any other endmember i is expressed as a function of the temperature and the partial pressure of oxygen via the equation

$$G_{i} = \Delta G_{i}^{0} - (T - T^{0}) \Delta S_{i}^{0} + \Delta C p_{i}^{0} \left(T - T^{0} - T ln(\frac{T}{T^{0}}) \right) - \Delta n_{i} \mu_{O_{2}}^{T, P_{O_{2}}}.$$
(1)

 G_i is set equal to the Gibbs free energy change in a reaction by which an endmember is obtained from an equivalent mixture of UO₂, $LnO_{1.5}$ (pyrochlore) and O₂ gas. Here ΔG_i^0 , ΔS_i^0 and ΔCp_i^0 are fitting parameters and Δn_i is the number of moles of O₂ gas consumed or added when the endmember i is built from a mixture of UO₂ and $LnO_{1.5}$. For example, the endmember $Ln_{1/2}U_{1/2}O_2$ of the fluorite solid solution is formed via the reaction

$$\frac{1}{2}UO_2 + \frac{1}{2}LnO_{1.5} + \frac{1}{8}O_2 = Ln_{1/2}U_{1/2}O_2.$$
 (2)

Thus, for $i = Ln_{1/2}U_{1/2}O_2 \Delta n_i = \frac{1}{8}$. Similarly, the endmember $Ln_{1/4}U_{3/4}O_{9/4}$ of the M₄O₉-type solid solution is obtained via the reaction

$$\frac{3}{4}UO_2 + \frac{1}{4}LnO_{1.5} + \frac{3}{16}O_2 = Ln_{1/4}U_{3/4}O_{9/4}.$$
 (3)

In this case $\Delta n_i = \frac{3}{16}$. The chemical potential of oxygen at a given temperature and a given partial pressure of O_2 is computed via the equation

$$\mu_{\mathcal{O}_2}^{T,P_{\mathcal{O}_2}} = -S_{\mathcal{O}_2}^0 \left(T - T^0 \right) + C p_{\mathcal{O}_2}^0 \left(T - T^0 - T \ln(\frac{T}{T^0}) \right) + RT \ln(P_{\mathcal{O}_2}/P^0), \tag{4}$$

where $S_{\rm O_2}^0 = 205.1373$ J/K/mol and $Cp_{\rm O_2}^0 = 29.355$ J/K/mol and where P^0 is the standard pressure of 101325 Pa, $T^0 = 298.15K^{60}$. Practically, the Gibbs free energy of an endmember and of a phase is made dependent not only on the temperature, but also on the partial pressure of oxygen. With this simplification modelling of the gas phase is not needed as its thermodynamic effect is included in the definition of the free energies of the solids.

The Gibbs free energy of a solid solution phase is described with a model that combines features of molecular mixing and sublattice models. The reference Gibbs free energy of a phase is modelled with the equation

$$G^{\text{ref}} = \sum_{i} X_i G_i,\tag{5}$$

where X_i is the endmember fraction. The endmembers as chemical components are thought to be split into molecular cation and anion entities, which are allowed to mix separately within sublattices. In this respect $\{U^4\}$, $\{Ln_{1/2}^3U_{1/2}^5\}$, $\{Ln_{2/3}^3U_{1/3}^6\}$, $\{U^5\}$ and $\{Ln^3\}$ are legitimate molecular cation entities of the fluorite phase, which can substitute each other within the cationic sublattice. The enthalpy of mixing within a sublattice is described with the regular mixing model

$$G^{\text{exess}} = \sum_{j \neq i} x_i x_j \Big(W_{ij}^{\text{h}} - T W_{ij}^{\text{s}} \Big), \tag{6}$$

where x_i is the fraction of molecular entity i within a sublattice. In a case of a binary solid solution, such as M_4O_9 or M_3O_8 , the mixing is assumed to occur only within the cationic sublattice, and the fractions of molecular species are set equal to the endmember fractions. The $x_i = X_i$ relation is also obtained in the case when enthalpic interactions are set to zero in the anionic sublattice. Such a model is adopted for fluorite. The configurational entropy is built from contributions from different sublattices as in CEF. But, when computing the entropy, the molecular entities, such as $\{Ln_{2/3}^3U_{1/3}^6\}$, are split into their elemental constituents. The free energy of a phase is described as follows

$$G^{\text{mix}} = G^{\text{ref}} + G^{\text{exess}} - TS^{\text{conf}},\tag{7}$$

where the last term combines entropic contributions from all sublattices. The free energy minimization is performed via a simple grid approach, where ich parameter is varied with a small increment over the whole parameter space. Special cases are discussed below.

Fluorite solid solution

The model of Ln-doped fluorite is improved relative to the previous study³² in two important aspects. First, an additional stoichiometric endmember, $U_{1/3}Ln_{2/3}O_2$ is introduced. This endmember, together with $U_{1/2}Ln_{1/2}O_2$, accounts for the tendency of vacancies and interstitials to annihilate, favouring the stoichiometric composition. While the $U_{1/2}Ln_{1/2}O_2$ endmember allows maintaining the stoichiometric relationship up to the limit of z = 0.5, the $U_{1/3}Ln_{2/3}O_2$ endmember, due to the presence of U^{+6} , allows extending the stoichiometric relation (O/M = 2) to z = 2/3. Second, the hypo- and hyper-stoichiometric limits are extended to δ = 0.5 and δ = 0.33, respectively. The upper limit is smaller than the theoretically possible value of δ = 0.5 ($X_{UO2.5}$ = 1) because of an additional SRO constraint, which is discussed below. UO_3 is excluded from the list of independent endmembers. This is justified by the observation that the O/M ratio in fluorite never exceeds 2.5.

The parameter space is built via a stepwise admixing of new components/endmembers to UO₂ fluorite as shown in Fig S7 (Supplementary materials). First, the $LnO_{1.5}$ component is added. A completely reduced solid solution is thus built of z moles of $LnO_{1.5}$ and 1-z moles of UO_2 giving the general formula of $U_{1-z}Ln_zO_{2-0.5z}$. Then, one mole of $LnO_{1.5}$ and one mole of UO_2 are allowed to react with 1/8 mol of O_2 producing two moles of the endmember $U_{1/2}Ln_{1/2}O_2$. If the reaction progress is denoted r, then the fraction of this endmember per mole of O_2 producing three moles of the endmember O_2 are allowed to react with one mole of O_2 and with 1/6 mol of O_2 producing three moles of the endmember O_2 and O_2 . If the reaction progress is denoted O_2 , then the fraction of this endmember is O_2 . The remaining fraction, O_3 of O_4 is O_4 and O_4 while the remaining fraction, O_4 of O_4 is O_4 and O_4 and O_4 are allowed to react with one mole of O_4 while the remaining fraction, O_4 of O_4 is O_4 and O_4 . The remaining fraction of O_4 is a simple function of O_4 and O_4 and O_4 is O_4 and O_4 . The non-stoichiometry parameter, O_4 is a simple function of the fractions of the endmembers O_4 and O_4 . The non-stoichiometry parameter, O_4 is a simple function of the fractions of the endmembers O_4 and O_4 .

The structural formula becomes $[U_{q-x}^4][Ln_y^3][Ln_r^3U_r^5][Ln_{2d}^3U_d^6][U_x^3]O_{2+0.5(x-y)}$, where the square brackets embrace species contributing to endmember fractions. With the notation "UO₂" = 1, " $LnO_{1.5}$ " = 2, "UO_{2.5}" = 3, "U_{1/2} $Ln_{1/2}O_2$ " = 4, "U_{1/3} $Ln_{2/3}O_2$ " = 5, the endmember fractions are given: $X_1 = 1 - z - r - d - x$, $X_2 = z - r - 2d$, $X_3 = x$, $X_4 = 2r$, and $X_5 = 3d$. These fractions appear in Eqns. 5 and 6. The variables r, d and

x are treated as variational parameters. Their equilibrium values are determined from the condition of the Gibbs free energy minimum.

The $LnO_{1.5}$ endmember is assumed to have the pyrochlore ($Ln_2Ln_2O_6VV$) structure. In this compound the cation coordination number is 6, and ¼ of oxygen sites is vacant. These vacant sites form an (empty) BCC sublattice within the oxygen lattice of a hypothetical $M_4(O,V)_8$ fluorite. Filling in all these sites gives the stoichiometry of MO_2 , where the M cation is eightfold coordinated. A partial filling produces 6-, 7- and eightfold coordinated cations. Thus, the UO_2 fluorite and the $LnO_{1.5}$ pyrochlore are logical endmember choices for a model in which the M cations adopt exclusively the coordination numbers 6, 7 and 8. A model with the latter constraint is more reasonable from energy grounds than, for example, a model of perfect randomness, where the M cation can adopt all coordination numbers between 0 and 8. Indeed, atomistic simulation studies have shown that vacancies in fluorite-type compounds due to the Coulombic repulsion tend avoiding each other at short near-neighbour distances 61,62 , making cation coordination numbers smaller than 6 much less probable compared to the random case. To emulate this repulsion and to exclude cation coordination numbers smaller than 6, the vacancy/oxygen mixing is restricted to a BCC sublattice consisting of a quarter of available oxygen lattice sites. As the fraction of vacancies over the whole oxygen lattice is 0.25y, within the sublattice it is y (i.e., four times larger). The entropy (per one mole of M cations) is given by the equation

$$S_{\text{O/V:F}}^{\text{conf}} = -(R/2)(y\ln(y) + (1-y)\ln(1-y)). \tag{8}$$

Similarly, the distribution of oxygen interstitials within vacant interstitial sites is assumed to be non-random. The O_i/V_i mixing is restricted to a sublattice consisting of 1/3 of the available interstitial sites. This is motivated by the observation that $UO_{2,33}$ (U_3O_7) represents a fluorite-related compound with the largest O/M ratio. As the concentration of interstitials over the whole interstitial lattice is 0.5x, it is three times larger within the sublattice. Thus, the entropy contribution due to the interstitials (per one mole of M cations) becomes

$$S_{\text{Oi/Vi:F}}^{\text{conf}} = -(R/3) \left(\left(\frac{3x}{2} \right) \ln \left(\frac{3x}{2} \right) + \left(1 - \frac{3x}{2} \right) \ln \left(1 - \frac{3x}{2} \right) \right). \tag{9}$$

Equation 9 limits the admissible fraction of the $UO_{2.5}$ component to x = 2/3.

The cations are assumed to be randomly mixed within the M site, however, the fraction, x, of U⁺⁵ cations is excluded from mixing with other cations. These are the U⁺⁵ cations that balance the excess negative charge of O⁻² interstitials. We assume that these U⁺⁵ cations are tightly associated to the interstitials and thus do not contribute to the configurational entropy. The other cations mix randomly with each other over the fraction 1 - x of M sites. The fractions of Ln^{+3} , U⁺⁴, U⁺⁵, and U⁺⁶ cations that are allowed to be mixed with each other are, thus, given by $t_1 = z/(1 - x)$, $t_2 = (q - x)/(1 - x)$, $t_3 = r/(1 - x)$ and $t_4 = d/(1 - x)$, respectively. The entropy equation is

$$S_{\text{M:F}}^{\text{conf}} = -(1 - x)R(t_1 \ln(t_1) + t_2 \ln(t_2) + t_3 \ln(t_3) + t_4 \ln(t_4)). \tag{10}$$

The entropy model could be compared to CEF. In CEF the structural formula of fluorite would be written $(M)_1(O)_{3/2}(O,V)_{1/2}(O/V)_{1/3}$. The difference is that in the present model a certain fraction of U^{+5} is excluded from mixing with other cations affecting the configurational entropy. The enthalpy of mixing is modelled differently from CEF. We assume that the excess effects within sublattices are due to interactions between *molecular* species which could be represented by combinations of ions with different charges. In the M sublattice these species are $\{U^4\}$, $\{Ln_{1/2}^3U_{1/2}^5\}$, $\{Ln_{2/3}^3U_{1/3}^6\}$, and $\{U^5\}$. In the anionic sublattices the species are O^{-2} and V. The mixing within the anionic sublattices is assumed athermal. Because of the latter assumption the model of mixing in fluorite is simply mapped onto the regular model of mixing of the five endmembers.

M₄O₉ solid solution

High-temperature galvanic-cell experiments on $\mathrm{UO_2\text{-}LaO_{1.5}}^{29}$ and $\mathrm{UO_2\text{-}LuO_{1.5}}^{63}$ systems showed that up to ~ 15 mol % of $LnO_{1.5}$ could be incorporated into the ordered $\mathrm{U_4O_9}$ phase at 1273 K. At a higher content of $LnO_{1.5}$ superlattice reflections disappeared²⁹. On the other hand, according to the data of Stadlbauer et al.²⁹, the lattice parameter of $\mathrm{U_{1.2}La_2O_{2.23}}$ samples increased linearly within the range of 0 < z < 0.3 showing no break at the order/disorder transition. Based on this observation, we postulate a solubility mechanism of the type $2\mathrm{U^{+4}} = Ln^{+3} + \mathrm{U^{+5}}$ in $\mathrm{M_4O_9}$ assuming the endmembers $\mathrm{U_{1/2}^4U_{1/2}^5O_{9/4}}$ and $\{Ln_{1/4}^3U_{1/4}^5\}\mathrm{U_{1/2}^5O_{9/4}}$ and a theoretical solubility limit of 25 mol % of $LnO_{1.5}$. In this model we assume the O/V arrangement within the interstitial site to be fully ordered. The configurational entropy is thus due to M cations only. As in the fluorite model, the $\mathrm{U^{+5}}$ cations that are needed for balancing $\mathrm{O^{-2}}$ interstitials are excluded from mixing with Ln^{+3} . This exclusion implies that the mixing of Ln^{+3} with U cations occurs over half of the M sites. If the Ln/M fraction in the solid solution is z, the fraction of $\{Ln_{1/4}^3U_{1/4}^5\}\mathrm{U_{1/2}^5}\mathrm{O_{9/4}}$ endmember is 4z. The structural formula is written $\{\mathrm{U_{(1-4z)/2}^4Ln_z^3U_z^5}\}\mathrm{U_{1/2}^5}\mathrm{O_{9/4}}$, where the curly brackets unite cations that are allowed to mix with each other. The configurational entropy (per one mole of M cations) is

$$S_{M,\gamma}^{\text{conf}} = -(R/2)(4z\ln(2z) + (1 - 4z)\ln(1 - 4z)). \tag{11}$$

The enthalpy of mixing is modelled using a single Margules parameter for the interaction between the endmembers $U_{1/2}^4 U_{1/2}^5 O_{9/4}$ and $\{Ln_{1/4}^3 U_{1/2}^5 O_{9/4} \cdot \text{The standard properties of the endmembers are defined according to the Eq. 1, relative to <math>UO_2$ fluorite and a hypothetical $LnO_{1.5}$ pyrochlore. The U_4O_9 phase exists in α -, β - and γ -forms. The structural transitions occur at ~323 K ($\alpha \leftrightarrow \beta$) and at ~873 K ($\beta \leftrightarrow \gamma$)⁶⁴. The present

modelling is mostly based on data corresponding to the stability of the high-temperature γ -form. However, as crystallographic differences between β - and γ -forms are minor⁶⁴, the developed solid solution model is thought applicable to the β -form as well.

M₃O₈ solid solution

At ambient temperatures U_3O_8 is orthorhombic $C2mm^{65}$. The phase is referred to as α -form. U^{+5} and U^{+6} cations occupy two different Wyckoff positions in the proportion 2:1. With the increase in the temperature at ~623 K the structure becomes hexagonal, $P\overline{6}2m$ (α '-form), where the cations occupy a single Wyckoff position. The α/α ' transition is fully reversible. At about 1173 K the hexagonal relationship $b/a = \sqrt{3}$ is destroyed, and the structure is again described as orthorhombic. When this high-temperature structure is slowly cooled down to room temperature, it is indexed in the Cmcm space group⁶⁶ and is known as β -form. The present study is mostly concerned with applications to ambient temperatures; thus, we develop models for the α and α' phases only. Few experimental $\log(P_{O_2}/P^0)$ vs. δ data that fall into the stability field of the β -form are fitted with the model for the α '-phase. This is justified by noting that the data do not show a strong variation across the α'/β transition. To fix the α/α transition at ~ 623 K, we assume that the entropy of the transformation is fully due to the configurational disorder of U^{+5} and U^{+6} within the single Wyckoff position of the α '-phase, which is 5.292 J/K/mol. The standard Gibbs free energy of the α-form is therefore set to be 3.3 kJ/mol more negative than the standard Gibbs free energy of the α'-phase, while the latter is fitted to experimental data. The solid solution models are developed assuming the $3U^{4.5} = Ln^{4.3} + 2U^{4.6}$ substitution mechanism 50,67. The solid solution in the α '-phase is modelled as a mixture of two endmembers, $\{U_{2/3}^5 U_{1/3}^6\}O_{8/3}$ and $\{Ln_{2/9}^3 U_{7/9}^6\}O_{8/3}$. If the Ln/M ratio in the phase is z, the fraction of the endmember $\{Ln_{2/9}^3 U_{7/9}^6\}O_{8/3}$ is 9z/2, the structural formula is $\{U_{2/3-3z}^5 Ln_z^3 U_{1/3+2z}^6\}O_{8/3}$. The entropy is given by the equation

$$S_{\alpha'-\text{U3O8}}^{\text{conf}} = -R\left(z\ln(z) + \left(\frac{1}{3} + 2z\right)\ln\left(\frac{1}{3} + 2z\right) + \left(\frac{2}{3} - 3z\right)\ln\left(\frac{2}{3} - 3z\right)\right). \tag{12}$$

In the low-temperature α -phase one Wyckoff position is fully filled by U⁺⁶. Thus, Ln-substitution is thought to occur only over 2/3 of M sites. The endmembers than have the composition of $U_{2/3}^5 U_{1/3}^6 O_{8/3}$ and $\{Ln_{2/9}^3 U_{4/9}^6\} U_{1/3}^6 O_{8/3}$. If the Ln/M ratio in the phase is z, the fraction of the $\{Ln_{2/9}^3 U_{4/9}^6\} U_{1/3}^6 O_{8/3}$ endmember is 9z/2, the structural formula becomes $\{U_{2/3-3z}^5 Ln_z^3 U_{2z}^6\} U_{1/3}^6 O_{8/3}$. The configurational entropy is written as

$$S_{\alpha-\text{U3O8}}^{\text{conf}} = -(2/3)R\left(\left(\frac{3z}{2}\right)\ln\left(\frac{3z}{2}\right) + (3z)\ln(3z) + \left(1 - \frac{9z}{2}\right)\ln\left(1 - \frac{9z}{2}\right)\right). \tag{13}$$

The enthalpy of mixing is defined by Margules parameters for the interactions between $\{U_{2/3}^5U_{1/3}^6\}O_{8/3}$ and $\{Ln_{2/9}^3U_{7/9}^6\}O_{8/3}$ in the α -phase and between $U_{2/3}^5U_{1/3}^6O_{8/3}$ and $\{Ln_{2/9}^3U_{4/9}^6\}U_{1/3}^6O_{8/3}$ in the α -phase.

Fitting strategy

Experimental data linking δ to T, z and $\log(P_{\rm O_2}/P^0)$ are most suitable for constraining the model of the fluorite phase. Here the data on pure UO₂-UO₃ ³⁶⁻³⁸ are used to determine the standard Gibbs free energy of UO₂ 5 (i.e., the ΔG_i^0 , ΔS_i^0 and $\Delta C p_i^0$ terms), as well as the Margules parameters (W_H and W_S) for the interaction between {U⁴} and {U⁵} species (equivalently between UO₂ and UO_{2.5} endmembers). The data for the GdO_{1.5}-UO₂-UO₃ system³⁶ are used for parameterizing the dependency of δ , on z, i.e., on the fraction of $LnO_{1.5}$. The minimum parameter set allowing this description includes the ΔG_i^0 , ΔS_i^0 and $\Delta C p_i^0$ values of the endmembers $U_{1/2} L n_{1/2} O_2$ and $U_{1/3}Ln_{2/3}O_2$, and the Margules parameter (W_H) for the interaction between $\{U^4\}$ and $\{Ln^3\}$. The initial values of all parameters were estimated from available thermodynamic data on UO_2 , γ - UO_3 and $U_3O_8^{68}$ with the additivity rule (see Table S1 in Supplementary materials). The data for UO₂ were set to zero values, while the data for γ -UO₃ and U₃O₈ were recalculated as increments over the data on UO₂. The standard properties of $LnO_{1.5}$ (pyrochlore) were set to zero. The parameters of U_4O_9 - and U_3O_8 -type phases were fitted to two-phase equilibrium data. Then the ΔG_i^0 and ΔS_i^0 of $\{U_{1/2}^4\}U_{1/2}^5Q_{9/4}^5$ were tuned such that the $U_4Q_9/UQ_{2+\delta}$ transition occurred at ~ 1373 K consistently with Saito³⁸. The ΔG_i^0 , ΔS_i^0 and ΔCp_i^0 parameters of the endmember $\{Ln_{1/4}^3U_{1/4}^5\}U_{1/2}^5O_{9/4}$ were fitted to the data of Stadlbauer et al. ²⁹. The ΔS_i^0 and $\Delta C p_i^0$ parameters of the $\{L n_{2/9}^3 U_{7/9}^4\} O_{8/3}$ endmember were estimated with the additivity rule. The ΔG_i^0 parameter of the endmember $\{L n_{2/9}^3 U_{7/9}^6\} O_{8/3}$ endmember and the Margules parameter for the interaction between $\{U^3\}$ and $\{L n_{3/3}^3 U_{2/3}^6\}$ species were adjusted such that at T>1173 K in air the model predicted a strong partitioning of $LnO_{1.5}$ into the fluorite phase, consistently with the high-temperature equilibrium data^{25–30}, while at T < 973 K the same model predicted the destabilization of fluorite relative to a *Ln*-rich M₃O₈ phase, consistently with the air oxidation experiments of Potts et al.⁵⁰.

Ion-packing model of fluorite-type phases

The lattice parameter of doped UO₂ fluorite varies as a function of composition, non-stoichiometry, and the type of Ln cation^{46–48}. These variations could be adequately modelled with the aid of an ion-packing model^{32,48,69}, which uses the geometrical relationship between the lattice parameter, a, and the sum of the average radii of cations, $\langle R_{\rm C} \rangle$, and anions, $\langle R_{\rm A} \rangle$

$$a = \frac{4}{\sqrt{3}} (\langle R_{\rm C} \rangle + \langle R_{\rm A} \rangle). \tag{14}$$

The coefficient $\frac{4}{\sqrt{3}}$ appears from the condition that cations and anions touch each other in [111] direction. The average radii of cations and anions are further evaluated as sums of radii of all cation and anion species weighted by their fractions in the structural formula $\{U_{q-x}^4Ln_z^3U_z^5V_d^6\}V_x^5\{O_{2-0.5y}V_{0.5y}\}O_{0.5x}$. The cations taken in the curly brackets split into fractions of 6-, 7-, and 8-fold coordinated species. The latter are calculated under the assumption that vacancies occur only within a BCC sublattice of the oxygen lattice of fluorite. As the fraction of vacancies in this sublattice is y, and as each cation is shared by two sites of the BCC sublattice, the fractions of 6-, 7-, and eightfold coordinated species are y^2 , 2y(1-y) and $(1-y)^2$, respectively. This means, for example, that U^{+4} cations occur in the solid solution as $U^{4,6}$, $U^{4,7}$, and $U^{4,8}$ species with the fractions $(q-x)y^2$, (q-x)2y(1-y) and $(q-x)(1-y)^2$, respectively. Here and below for cation species the notation $M^{Q,K}$ is used, where Q and K are the charge and the coordination number, respectively. The U^{+5} cations that are not included in the curly brackets in the structural formulae are assumed to have a coordination number larger than 8. These cations balance the excess oxygen charge and are assumed to have one or two interstitial anions in their close neighbourhood. These species is denoted with the symbol $U^{5,9}$. The average radius of an anion is composed of the contributions from the lattice oxygen O^{-2} and the oxygen vacancy

$$\langle R_{\rm A} \rangle = (1 - 0.25y)R_{\rm O} + 0.25yR_{\rm V}.$$
 (15)

Following the studies of Bukaemskiy et al.⁶⁹ and Vinograd et al.³², we assume that a vacancy has a defined radius that is about 12% larger than the ionic radius of oxygen. The values of $R_{\rm O}=1.3736$ Å and $R_{\rm V}=1.5410$ Å are adopted from previous studies^{32,69}. The oxygen interstitials are assumed to have zero ionic radius, while their effect on the lattice parameter is included into the effective size of the U^{5,9} cation. Because the lattice contracts upon the oxidation from MO₂ to MO₂₊₆, the ionic radius of U^{5,9} is set smaller than the radius of U^{4,8}. The radii of U^{4,8}, U^{4,7}, U^{4,6}, U^{5,8}, U^{5,7}, U^{5,6}, $Ln^{3,8}$, $Ln^{3,7}$, $Ln^{3,6}$ are adopted with small modifications from ³². The radius of U^{6,8} is fitted to the lattice parameter data on fully oxidised UO₂-LnO_{1,5} solid solutions with the composition of $z \sim 0.67^{25,26}$ under the assumption that at these conditions Ln and U occur exclusively as $Ln^{3,8}$ and U^{6,8}. This assumption is justified in the Results section. The radii of U^{6,7} and U^{6,6} are constrained under the assumption that the radii of U^{Q,K} vary linearly with K, while the slope of $R_{\rm U}^{Q,K}$ vs. K is the same for $Q = \{4, 5 \text{ and } 6\}$. The radii are given in Table S2 (Supplementary materials).

The lattice parameter of the M_4O_9 phase is also modelled with Eqns. 14 and 15. The structural formula of M_4O_9 can be written as $\{U^4_{(1-4z)/2}Ln_z^3U_z^5\}U^5_{1/2}O_2O_{1/4}$. There are no vacancies, thus, all cations that are embraced by curly brackets occur in eightfold coordination, while the average anion radius is equal to the ionic radius of O^{-2} .

Data availability

The datasets generated during and/or analysed during the current study are available from the corresponding author on reasonable request.

Received: 15 February 2023; Accepted: 12 September 2023

Published online: 20 October 2023

References

- Kleykamp, H. The chemical state of fission products in oxide fuels at different stages of the nuclear fuel cycle. Nucl. Tech. 80, 412-422 (1988).
- 2. Carbol, P., Wegen, D. H., Wiss, T. & Fors, P. Spent nuclear fuel as waste material in *Comprehensive Nuclear Materials* (ed. Konings, R.) 389–420 (Elsevier, 2012).
- 3. Böhm, W., Kiehlmann, H. D., Neufert, A. & Peehs, M. Gd₂O₃ up to 9 weight percent, an established burnable poison for advanced fuel management in pressurized water reactors. *Kerntechnik* **50**, 234–240 (1987).
- Kim, J.-S., Kwon, S.-K., Sanchez, M. & Cho, G.-C. Geological storage of high-level nuclear waste. KSCE J. Civ. Eng. 15, 721–737. https://doi.org/10.1007/s12205-011-0012-8 (2011).
- Kertes, A. S. & Guillaumont, R. Solubility of UO₂. A comparative review. *Nucl. Chem. Waste Manag.* 5, 215–219. https://doi.org/10.1016/0191-815X(85)90080-4 (1985).
- 6. Ewing, R. C. Long-term storage of spent nuclear fuel. Nat. Mat. 14, 252-257. https://doi.org/10.1038/nmat4226 (2015).
- 7. Trummer, M., Dahlgren, B. & Jonsson, M. The effect of Y₂O₃ on the dynamics of oxidative dissolution of UO₂. *J. Nucl. Mater.* **407**, 195–199. https://doi.org/10.1016/j.jnucmat.2010.10.014 (2010).
- 8. Pehrman, R., Trummer, M., Lousada, C. M. & Jonsson, M. On the redox reactivity of doped UO₂ pellets—influence of dopants on the H₂O₂ decomposition mechanism. *J. Nucl. Mater.* **430**, 6–11. https://doi.org/10.1016/j.jnucmat.2012.06.016 (2012).
- Barreiro Fidalgo, A. & Jonsson, M. Radiation induced dissolution of (U, Gd)O₂ pellets in aqueous solution—A comparison to standard UO₂ pellets. J. Nucl. Mater. 514, 216–223. https://doi.org/10.1016/j.jnucmat.2018.11.037 (2019).
- He, H., Keech, P., Broczkowski, M., Noël, J. & Shoesmith, D. Characterization of the influence of fission product doping on the anodic reactivity of uranium dioxide. Can. J. Chem. 85, 702–713. https://doi.org/10.1139/v07-056 (2007).
- 11. Razdan, M. & Shoesmith, D. W. The electrochemical reactivity of 6.0 wt% Gd-doped UO₂ in aqueous carbonate/bicarbonate solutions. *J. Electrochem. Soc.* **161**, H225–H234. https://doi.org/10.1149/2.050404jes (2014).
- Liu, N., He, H., Noël, J. J. & Shoesmith, D. W. The electrochemical study of Dy₂O₃ doped UO₂ in slightly alkaline sodium carbonate/ bicarbonate and phosphate solutions. *Electrochim. Acta.* 235, 654–663. https://doi.org/10.1016/j.electacta.2017.03.075 (2017).
- Liu, N. Z. et al. Influence of Gd doping on the structure and electrochemical behavior of UO₂. Electrochim. Acta 247, 496–504. https://doi.org/10.1016/j.electacta.2017.07.006 (2017).
- 14. Thomas, L. E., Einziger, R. E. & Buchanan, R. E. Effect of fission products on air oxidation of LWR spent fuel. *J. Nucl. Mater.* 201, 310–319. https://doi.org/10.1016/0022-3115(93)90187-4 (1993).
- 15. Cobos, J., Papaioannou, D., Spino, J. & Coquerelle, M. Phase characterisation of simulated high burn-up UO₂ fuel. *J. Alloys and Compounds* 271–273, 610–615. https://doi.org/10.1016/S0925-8388(98)00170-4 (1998).
- Kim, J.-G., Ha, Y.-K., Park, S.-D., Jee, K.-Y. & Kim, W.-H. Effect of a trivalent dopant, Gd³⁺, on the oxidation of uranium dioxide. J. Nucl. Mater. 297, 327–331. https://doi.org/10.1016/S0022-3115(01)00639-0 (2001).
- Scheele, R. D., Hanson, B. D. & Casella, A. M. Effect of added gadolinium oxide on the thermal air oxidation of uranium dioxide. J. Nucl. Mater. 552, 153008. https://doi.org/10.1016/j.jnucmat.2021.153008 (2021).

- 18. Talip, Z. et al. Raman and X-ray studies of uranium-lanthanum-mixed oxides before and after air oxidation. J. Am. Ceram. Soc. 98, 2278–2285. https://doi.org/10.1111/jace.13559 (2015).
- 19. Spahiu, K. State of the Knowledge Report Spent nuclear fuel (Domain 3.1.1) Preprint at https://www.ejp-eurad.eu/publications/state-knowledge-report-spent-nuclear-fuel-domain-311-doi-105281zenodo7024752. https://doi.org/10.5281/zenodo.7024752
- Solomon, J. M., Navrotsky, A. & Asta, M. Energetics and defect clustering trends for trivalent rare earth cations substituted in UO₂.
 J. Nucl. Mater. 457, 252–255. https://doi.org/10.1016/j.jnucmat.2014.11.089 (2015).
- Casella, A., Hanson, B. & Miller, W. The effect of fuel chemistry on UO₂ dissolution. J. Nucl. Mater. 476, 45–55. https://doi.org/10. 1016/j.jnucmat.2016.04.025 (2016).
- Grandstaff, D. E. A kinetic study of the dissolution of uraninite. Econ. Geol. 71, 1493–1506. https://doi.org/10.2113/gsecongeo. 71.8.1493 (1976).
- 23. Mazeina, L., Navrotsky, A. & Greenblatt, M. Calorimetric determination of energetics of solid solutions of UO_{2+x} with CaO and Y₂O₃. *J. Nucl. Mater.* **373**, 39–43. https://doi.org/10.1016/j.jnucmat.2007.03.269 (2008).
- 24. Zhang, L. & Navrotsky, A. Thermochemistry of rare earth doped uranium oxides LnxU1-xO2-0.5+y (Ln = La, Y, Nd). J. Nucl. Mater. 465, 682-691. https://doi.org/10.1016/j.jnucmat.2015.06.059 (2015).
- 25. Diehl, H. G. & Keller, C. Das system UO₂-UO₃-LaO_{1.5}. *J. Solid State Chem.* **3**, 621–636. https://doi.org/10.1016/0022-4596(71) 90110-1 (1971).
- Keller, C. & Boroujerdi, A. Phasengleichgewichte in den Systemen UO₂-UO₃-NdO_{1.5} und NpO_{2+x}-NdO_{1.5}. J. Inorg. Nucl. Chem. 34, 1187–1193. https://doi.org/10.1016/0022-1902(72)80318-X (1972).
- Berndt, U., Tanamas, R. & Keller, C. The ternary UO₂-UO₃-EuO_{1.5}(EuO) system and investigation of Eu(II)-actinide(IV)-perovskites. J. Solid State. Chem. 17, 113–120. https://doi.org/10.1016/0022-4596(76)90209-7 (1976).
- De Alleluia, I. B., Hoshi, M., Jocher, W. G. & Keller, C. Phase relationships for the ternary UO₂-UO₃-REO_{1.5} (RE = Pr, Nd, Dy). J. Inorg. Nucl. Chem. 43, 1831–1834. https://doi.org/10.1016/0022-1902(81)80392-2 (1981).
- Stadlbauer, E., Wichmann, U., Lott, U. & Keller, C. Thermodynamics and phase relationships of the ternary lanthanum-uranium-oxygen system. J. Solid State Chem. 10, 341–350. https://doi.org/10.1016/0022-4596(74)90043-7 (1974).
- 30. Keller, C., Engerer, H., Leitner, L. & Sriyotha, U. Phasengleichgewichte in den Systemen Uranoxid-Oxid der seltenen Erden (Ho, Er, Tm, Yb, Lu). J. Inorg. Nucl. Chem. 31, 965–980. https://doi.org/10.1016/0022-1902(69)80144-2 (1969).
- 31. Roberts, L. E. J. The actinide oxides. Q. Rev. Chem. Soc. 15, 442-460. https://doi.org/10.1039/QR9611500442 (1961).
- 32. Vinograd, V. L., Bukaemskiy, A. A., Modolo, G., Deissmann, G. & Bosbach, D. Thermodynamic and structural modelling of non-stoichiometric Ln-doped UO₂ solid solutions, Ln = {La, Pr, Nd, Gd}. Front. Chem. 9, 705024. https://doi.org/10.3389/fchem.2021. 705024 (2021).
- 33. Une, K. & Oguma, M. Oxygen potentials of (U, Nd)O_{2±}x solid solutions in the temperature range 1000–1500°C. *J. Nucl. Mater.* 118, 189–194. https://doi.org/10.1016/0022-3115(83)90224-6 (1983).
- Yoshida, K. et al. Oxygen potential of hypo-stoichiometric La-doped UO₂. J. Nucl. Mater. 418, 22–26. https://doi.org/10.1016/j.jnucmat.2011.06.045 (2011).
- 35. Hagemark, K. & Broli, M. Equilibrium oxygen pressures over solid solutions of urania-yttria and urania-lanthana at 1100° to 1400°C. *J. Am. Ceram. Soc.* 50, 563–567. https://doi.org/10.1111/j.1151-2916.1967.tb14999.x (1967).
- Lindemer, T. B. & Sutton, A. L. Jr. Study of nonstoichiometry of Ú_{1-z}Gd_zO_{2±x}. J. Am. Ceram. Soc. 71, 553-561. https://doi.org/10. 1111/j.1151-2916.1988.tb05919.x (1988).
- Nakamura, A. & Fujino, T. Thermodynamic study of UO_{2+x} by solid state emf technique. J. Nucl. Mater. 149, 80–100. https://doi. org/10.1016/0022-3115(87)90501-0 (1987).
- 38. Saito, Y. Nonstoichiometry in uranium dioxide. J. Nucl. Mater. 51, 112–125. https://doi.org/10.1016/0022-3115(74)90121-4 (1974).
- 39. Kuikkola, K. et al. High-temperature electrochemical study of uranium oxides in the UO₂-U₃O₈ region. Acta Chem. Scand. 16, 327–345. https://doi.org/10.3891/acta.chem.scand.16-0327 (1962).
- Leinders, G. et al. Assessment of the U₃O₇ crystal structure by X-ray and electron diffraction. Inorg. Chem. 55, 9923–9936. https://doi.org/10.1021/acs.inorgchem.6b01941 (2016).
- 41. Grenthe, I., Fuger, J., Konings, R.J.M., Lemire, R.J., Muller, A.B., Nguyen-Trung, C., Wanner, H. Chemical Thermodynamics of Uranium, NEA/OECD and North-Holland, 1992.
- Beals, R. J. & Handwerk, J. H. Solid solutions in the system Urania—rare-earth oxides: I, UO₂-GdO_{1.5}. J. Am. Ceram. Soc. 48, 271–274. https://doi.org/10.1111/j.1151-2916.1965.tb14735.x (1965).
- 43. Bartram, S. F., Juenke, E. F. & Aitken, E. A. Phase relations in the system UO₂-UO₃-Y₂O₃. J. Amer. Ceram. Soc. 47, 171–175. https://doi.org/10.1111/j.1151-2916.1964.tb14386.x (1964).
- 44. Shannon, R. Revised effective ionic radii and systematic studies of interatomic distances in halides and chalcogenides. *Acta Cryst.* **A32**, 751–767. https://doi.org/10.1107/S0567739476001551 (1976).
- 45. Wadier, J. -F. Phase diagrams and thermodynamic properties of the uranium neodymium-oxygen system. Tech. rep. Rapport CEA-R-4507, Commisariat à l'énergie atomique (1973)
- 46. Lee, S. M., Knight, T. W., Voit, S. L. & Barabash, R. I. Lattice parameter behavior with different Nd and O concentrations in $(U_{1-v}Nd_v)O_{2+x}$ solid solution. *Nucl. Technol.* 193, 287–296 (2016).
- 47. Fukushima, S., Ohmichi, T., Maeda, A. & Handa, M. Thermal conductivity of near-stoichiometric (U, Nd)O₂, (U, Sm)O₂ and (U, Eu)O₂ solid solutions. *J. Nucl. Mater.* 114, 312–325. https://doi.org/10.1016/0022-3115(83)90270-2 (1983).
- 48. Ohmichi, T., Fukushima, S., Maeda, A. & Watanabe, H. On the relation between lattice parameter and O/M ratio for uranium dioxide-trivalent rare earth oxide solid solution. *J. Nucl. Mater.* **102**, 40–66. https://doi.org/10.1016/0022-3115(81)90544-4 (1981).
- 49. Zhang, L., Solomon, J. M., Asta, M. & Navrotsky, A. A combined calorimetric and computational study of the energetics of rare earth substituted UO₂ systems. *Acta Mater.* **97**, 191–198. https://doi.org/10.1016/j.actamat.2015.06.048 (2015).
- 50. Potts, S. K. *et al.* Structural incorporation of lanthanides (La, Eu, and Lu) into U₃O₈ as a function of the ionic radius. *MRS Adv.* 7, 128–133. https://doi.org/10.1557/s43580-022-00226-1 (2022).
- 51. Matzke, H. Atomic transport properties in UO_2 and mixed oxides $(U, Pu)O_2$. Chem. Soc. Faraday Trans. 2(83), 1121–1142. https://
- doi.org/10.1039/F29878301121 (1987).
 Perriot, R., Liu, X.-Y., Stanek, C. R. & Andersson, D. A. Diffusion of Zr, Ru, Ce, Y, La, Sr and Ba fission products in UO₂. J. Nucl. Mater. 459, 90–96. https://doi.org/10.1016/j.jnucmat.2015.01.001 (2015).
- 53. Kerleguer, V. et al. The mechanisms of alteration of a homogeneous U_{0.73}Pu_{0.27}O₂ MOx fuel under alpha radiolysis of water. J. Nucl. Mater. **529**, 151920. https://doi.org/10.1016/j.jnucmat.2019.151920 (2020).
- 54. Cakir, P., Eloirdi, R., Huber, F., Konings, R. J. M. & Gouder, T. Thorium effect on the oxidation of uranium: Photoelectron spectroscopy (XPS/UPS) and cyclic voltammetry (CV) investigation on (U_{1-x}Th_x)O₂ (x = 0 to 1) thin films. *Appl. Surf. Sci.* **393**, 204–211. https://doi.org/10.1016/j.apsusc.2016.10.010 (2017).
- 55. McMurray, J. W., Shin, D. & Besmann, T. M. Thermodynamic assessment of the U-La-O system. *J. Nucl. Mater.* **456**, 142–150. https://doi.org/10.1016/j.jnucmat.2014.09.031 (2015).
- 56. Lee, S. M., Knight, T. W., McMurray, J. W. & Besmann, T. M. Measurement of the oxygen partial pressure and thermodynamic modeling of the U-Nd-O system. J. Nucl. Mater. 473, 272–282. https://doi.org/10.1016/j.jnucmat.2016.02.024 (2016).
- 57. Dottavio, G., Pontillon, Y., Desgranges, L., Guéneau, C. & Belin, R. C. Characterising the U-Nd-O miscibility gap by an experimental and a theoretical approach. *J. Nucl. Mater.* 458, 394–405. https://doi.org/10.1016/j.jnucmat.2014.12.061 (2015).

- 58. Hillert, M. The compound energy formalism. J. Alloy Compd. 320, 161–176. https://doi.org/10.1016/S0925-8388(00)01481-X (2001).
- Hallstedt, B. Thermodynamic assessment of the system MgO-Al₂O₃. J. Am. Ceram. Soc. 75, 1497–1507. https://doi.org/10.1111/j. 1151-2916.1992.tb04216.x (1992).
- 60. Finnis, M. W., Lozovoi, A. Y. & Alavi, A. The oxidation of NiAl: What can we learn from ab initio calculations?. *Ann. Rev. Mater. Res.* 35, 167–207. https://doi.org/10.1146/annurev.matsci.35.101503.091652 (2005).
- Bogicevic, A., Wolverton, C., Crosbie, G. & Stechel, E. Defect ordering in aliovalently doped cubic zirconia from first principles. *Phys. Rev. B* 64, 014106. https://doi.org/10.1103/PhysRevB.64.014106 (2001).
- 62. Bogicevic, A. & Wolverton, C. Nature and strength of defect interactions in cubic stabilized zirconia. *Phys. Rev. B* 67, 024106. https://doi.org/10.1103/PhysRevB.67.024106 (2003).
- 63. Wichmann, U., Berndt, U. & Keller, C. Phase equilibria and thermodynamic data for the ternary UO₂-UO₃-REO_{1.5} (Ho, Tm, Lu) systems. *Rev. Chim. Minérale* 14, 105–118 (1977).
- 64. Desgranges, L., Baldinozzi, G., Simeone, D. & Fischer, H. E. Structural changes in the local environment of uranium atoms in the three phases of U₄O₉. *Inorg. Chem.* **55**, 7485–7491. https://doi.org/10.1021/acs.inorgchem.6b00654 (2016).
- Ackermann, R., Chang, A. & Sorrell, C. A. Thermal expansion and phase transformations of the U₃O_{8-z} phase in air. J. Inorg. Nucl. Chem. 39, 75–85. https://doi.org/10.1016/0022-1902(77)80436-3 (1977).
- 66. Loopstra, B. O. Structure of beta-U₃O₈. Acta Cryst. B26, 656-657. https://doi.org/10.1107/S0567740870002935 (1970).
- Caisso, M. et al. Evidence of trivalent Am substitution into U₃O₈. Inorg. Chem. 55, 10438–10444. https://doi.org/10.1107/S1600 577520014265 (2016).
- Chartier, A., Van Brutzel, L., Fossati, P., Martin, Ph., and Guéneau, Chr. Thermodynamic and Thermophysical Properties of the Actinide Oxides. In Comprechensive Nuclear Materials (Second Edition) Vol. 7 (eds Konings, R. J. M., Stoller, R.E. and Agarwal, R.) 111–154 (Elsevier, 2020). https://doi.org/10.1016/B978-0-12-803581-8.11786-2
- Bukaemskiy, A. A., Vinograd, V. L. & Kowalski, P. M. Ion distribution models for defect fluorite ZrO₂-AO_{1.5} (A = Ln, Y) solid solutions: I. Relationship between lattice parameter and composition. *Acta Mater.* 202, 99–111. https://doi.org/10.1016/j.actam at.2020.10.045 (2021).

Acknowledgements

The study benefited from discussions with D. Bosbach, G. L. Murphy, S. Neumeier, S. K. Potts, P. Kegler, C. Schreinemachers, E. Yazhenskikh, G. Leinders, R. Phatak, D. Kulik, E. Curti, O. Fabrichnaya, S. Divinski and B. Hallstedt.

Funding

Open Access funding enabled and organized by Projekt DEAL.

Competing interests

The authors declare no competing interests.

Additional information

Supplementary Information The online version contains supplementary material available at https://doi.org/10.1038/s41598-023-42616-x.

Correspondence and requests for materials should be addressed to V.L.V.

Reprints and permissions information is available at www.nature.com/reprints.

Publisher's note Springer Nature remains neutral with regard to jurisdictional claims in published maps and institutional affiliations.

Open Access This article is licensed under a Creative Commons Attribution 4.0 International License, which permits use, sharing, adaptation, distribution and reproduction in any medium or format, as long as you give appropriate credit to the original author(s) and the source, provide a link to the Creative Commons licence, and indicate if changes were made. The images or other third party material in this article are included in the article's Creative Commons licence, unless indicated otherwise in a credit line to the material. If material is not included in the article's Creative Commons licence and your intended use is not permitted by statutory regulation or exceeds the permitted use, you will need to obtain permission directly from the copyright holder. To view a copy of this licence, visit http://creativecommons.org/licenses/by/4.0/.

© The Author(s) 2023